Design of the ITER Magnets to Provide Plasma Operational Flexibility

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(with slide contributions from J. Minervini, P. Lee, I. Rodin, P. Bruzzone)



Developments since 2001 (ITER Final Design Report)

Problems in 2001.....

Design in 2001 had a significant number of open options eg

3 conductor/layouts for the CS, 2 for the TF

2 concepts for the TF structures

Preparation of manufacturing specifications made difficult due to design uncertainty.

.....Solutions in 2004

Oetailed investigations, negotiations with the ITER partners
 Single agreed reference design.

@Detailed design and analysis (mostly in the inner poloidal key region, in the TF cooling and nuclear shielding and the CCs)
 → FLEXIBILITY to operate with a range of plasmas and cover uncertainties (in control, nuclear heating, plasma parameters)
 @No change to the cost or overall machine parameters

Limited time for talk \rightarrow pick out 4 examples in more detail

Main Work Areas

Design improvements to improve functionality Inner Poloidal Keys Outer OIS (friction joint) Central solenoid layout

Response to R&D results Nb3Sn conductor

Optimisation to reduce costs
TF Case fabrication route
Coil and structure cooling

Definition of Critical Components for Manufacturing
Correction coils

→ select 4 limited examples for presentation

Structural Design of TF Coil Case

>3 Material classes to minimise weight of high strength high cost steel

>Segmentation to improve fit of winding in case, easier closure welds

>Optimisation of poloidal key ways to distribute loads evenly

Structural design to include defects and fatigue (ASME XI procedures)
 SN and initial defect limits
 Keyways typical of SN limit
 Case typical of defect limit

>Leave margin for effect of tolerances and misalignments



Flux Optimisation of the Central Solenoid

Central solenoid has conflicting requirements
 Flux generation to drive plasma
 Minimum space to shrink machine size
 Shaping function for outer part (uniform current density not possible)

◆Shaping function has large impact
 →vertical support structure to hold CS together
 →Independent current supplies to modules

CS has been adjusted to lower field (13.5T→ 13T) since FDR2001
 Lower thickness (lower stress, less Nb3Sn) allows same flux, lower cost

Integration of outer current feeders and vertical support, less radial space

Cooling brought into inner bore

Identical modules in stack for redundancy (1 spare for all)

Central Solenoid

Total Weight of Nb3Sn Strands (t) Prebias Flux (Vs)





Redesign of TF Conductor

@Improvements in strand performance since 1994 (model coils)
 → ITER action in 2002 to re-assess industrial availability
 All 6 ITER PTs now in pre-qualification action for strand supply
 ◆Confirmed that strand specification can be increased at least to ITER proposed values

@Assessment of ITER model coils showed conductor performance less than expected, also evidence of performance drop of s/c dependent on 'transverse load' (BI force)

@Caused by local bending of strands, current degradation of strain sensitive Nb3Sn and in some cases local filament fracture

Correction by

Indecreased void fraction to improve strand support

▶steel jacket to give overall compression, reduce number of filaments going into tension

high performance strand to increase margins to allow for degradation
 Limit currents (and BI forces) in individual strands

Published Range of Strand Performance for ITER 1994-2004

lowest-mean (weight averaged)-maximum jc



Critical Current Density 12T 4.2K A/mm2

Oxford Instruments (US strand contract)

- 19 subelements
- single Ta barrier
- Cu:non-Cu = 1
- billet size ~35 kg
 > 3 billets are expected to fill 100 kg requirement
- Production billets would be larger (60 kg+)







Assessment of TFMC Results – Current Sharing Temperature Relative to 'Expected' Value, against Local Magnetic Load BI



Sectioning of the TF Insert Conductor (Ti jckt) After Operation



(I Rodin, Efremov Lab)

Cable compressed permanently to one side

TFI Cable Before and After Operation



The mean number of the defects per 1 m of length of last stage subcable is 223:

- 172 are placed in the wraping zone;

-51 are placed on the strands directly



1&2 Last stage subcables before the heat-treatment and test

3 -Last stage subcable after the heat-treatment and test

4. Ti spiral before the heat-treatment and test

In operation strands pressed into central cooling channel





Sectioning of a strand after 0.6% bending

(University of Wisconsin, P.Lee)

PIT Powder-in-tube Nb(Ta) at 0.6%



Nb3Sn Behaviour in Conductors

Sultan short sample test of Steel and Ti jacket, identical cable 1/6 ITER scale



Coil cooling optimisation and cryoplant control

ITER thermal load variable -> primary cooling circuit buffers cryoplant

>Cryoplant heat load smoothed by buffering heat in coils
 → limits on pulse rate of ITER especially in H operation when loads may be unexpected (ie disruptions, control)
 → Pulse schedule needs to be planned to match cryoplant

>Two conflicting requirements

Thermal loads (operating cost) depend on pump power, current leads

@Conductor design (construction cost) depends on AC losses,
 nuclear heating

Higher pump loads → less superconductor, lower construction cost

>Design optimisation of conductor with

Central cooling channel to reduce pressure drop (and pump power)
 Minimum length cooling channels compatible with winding
 Optimised He inlets (low pressure drop)





Contributions to Cryoplant Load and Distribution Over 1800s Reference Pulse with 500MW nuclear power (current leads He consumption converted to Joules with 1I/hr=6W)



SUMMARY OF NUCLEAR HEAT LOADS AND FLEXIBILITY OF COOLING

	Normal Operation (allow 33% margin on nuclear heat)	Operation with 100% Excess Nuclear Heat
Total nuclear heat (during burn) kW	13.9	18
Mass flow rate in TF thermal screen/ winding kg/s	4.5 / 2	6/3
TF pump heating power, 70% efficiency, kW, thermal screen /winding	1.0 /2.6	2.2 /5.6
Pulse rate to maintain constant average heat load	1 every 30 mins	1 every 45 mins

Uncertainty in nuclear heating (due to nuclear data and blanket assembly gaps) → Margin (up to 100% uncertainty) by adjusting pulse rate of 2/hour

Conclusions ---- FAQs

Are there basic questions over the feasibility and performance of the magnets? No

Is R&D needed before magnet construction can start Yes

We do not fully understand reasons for strand-in-cable degradation → Must qualify conductor BEFORE fabrication by short sample test (and do supporting R&D to improve understanding) → Need industrial input for optimisation of structure fabrication

→ Need industrial development on insulation, precompression rings

What else has to be done before PTs can start to place procurement contracts Lots of supporting design and analysis (FE stresses, cooling simulation etc) We must avoid design iterations once PTs start procurement...too many interfaces to control

No major design changes but adjustments within individual components

What is the soonest procurement (orders for Nb3Sn strand) could start After qualification tests on conductor samples...Probably Nov 2005

What is limiting progress on the magnet design Effort available to IT to work on main issues

ITER Magnet Construction

Time Schedule for Strand

