

Overview of recent work on carbon erosion, migration and long-term fuel retention in the EU-fusion programme and conclusions for ITER

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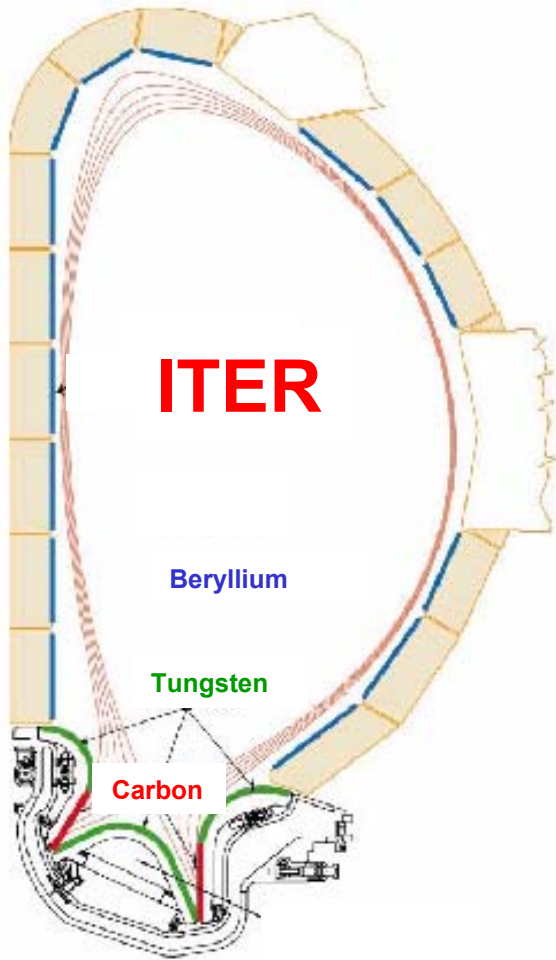
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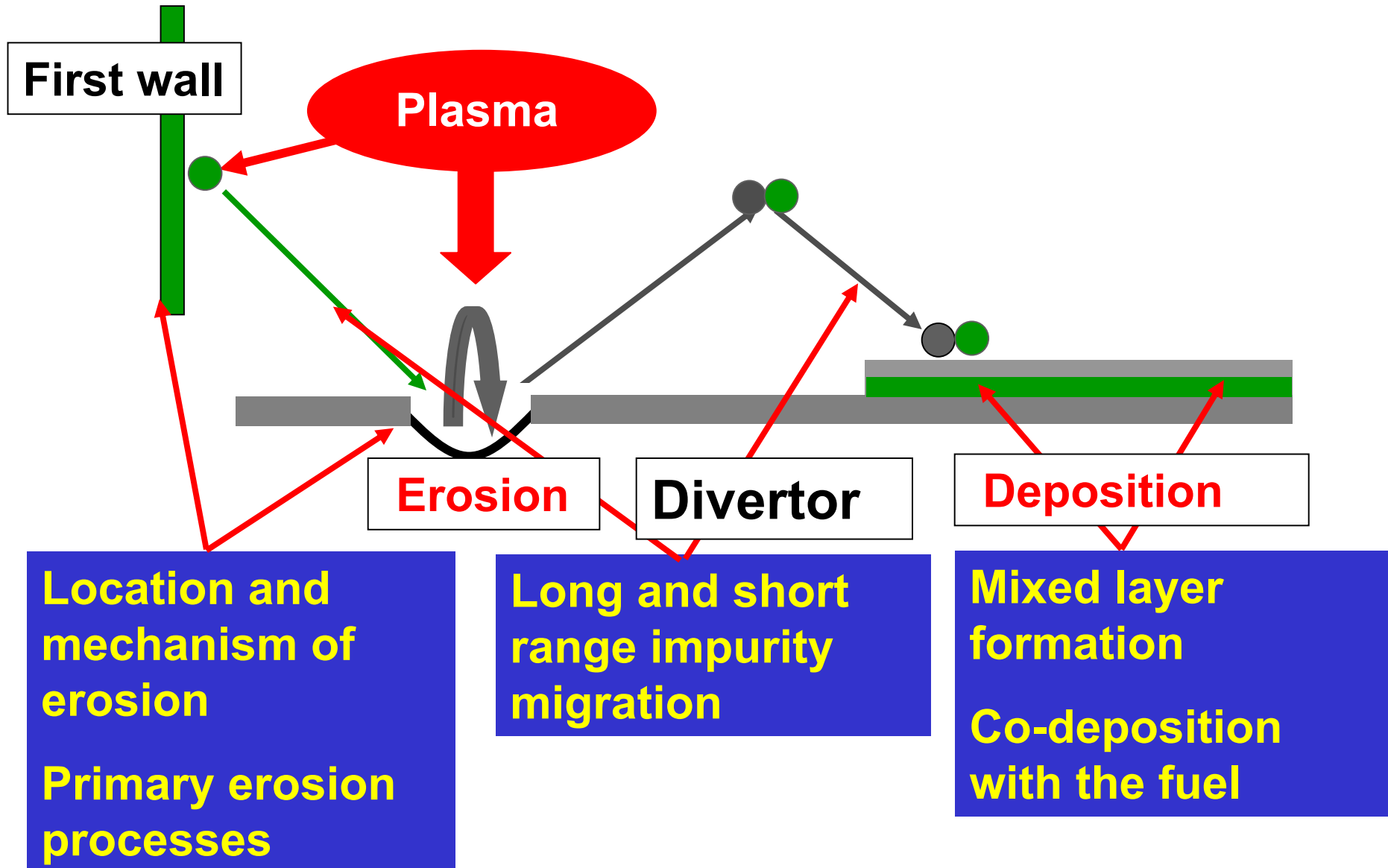
Long term Tritium inventory and lifetime of plasma facing components are most critical questions for ITER and future power plants

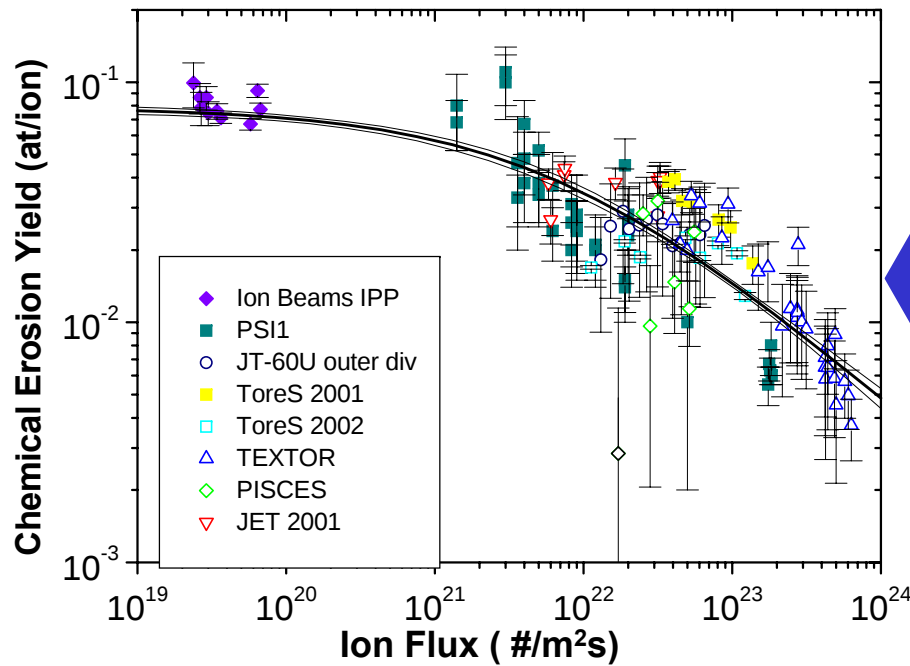
Understanding and development of control schemes is the major topic of the EU Task Force on Plasma Wall Interaction (related with the present ITER wall material choice)

in parallel:

The full tungsten wall programme in ASDEX-U → following talk by R.Neu

Processes that determine tritium inventory





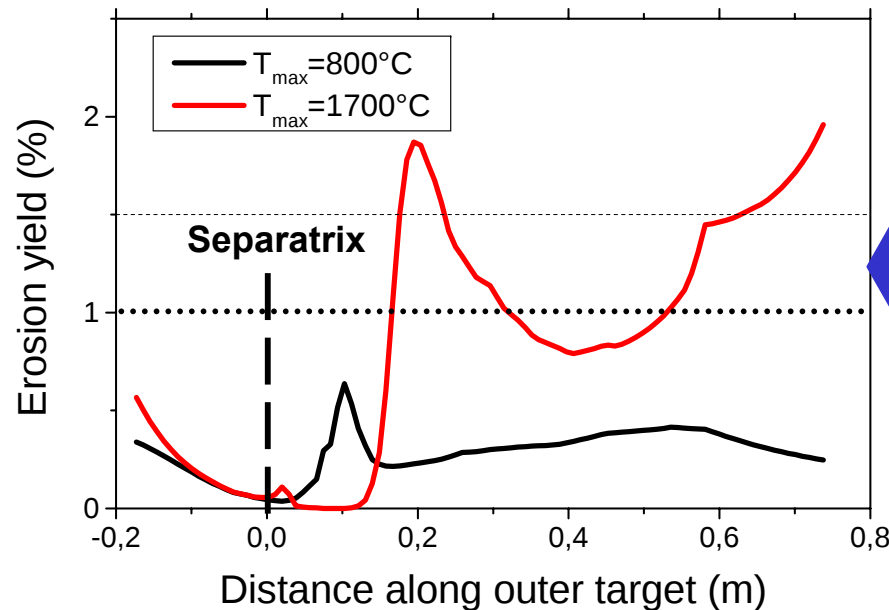
Normalisation of chemical erosion data

- Ion impact energy
- Surface temperature
- Flux

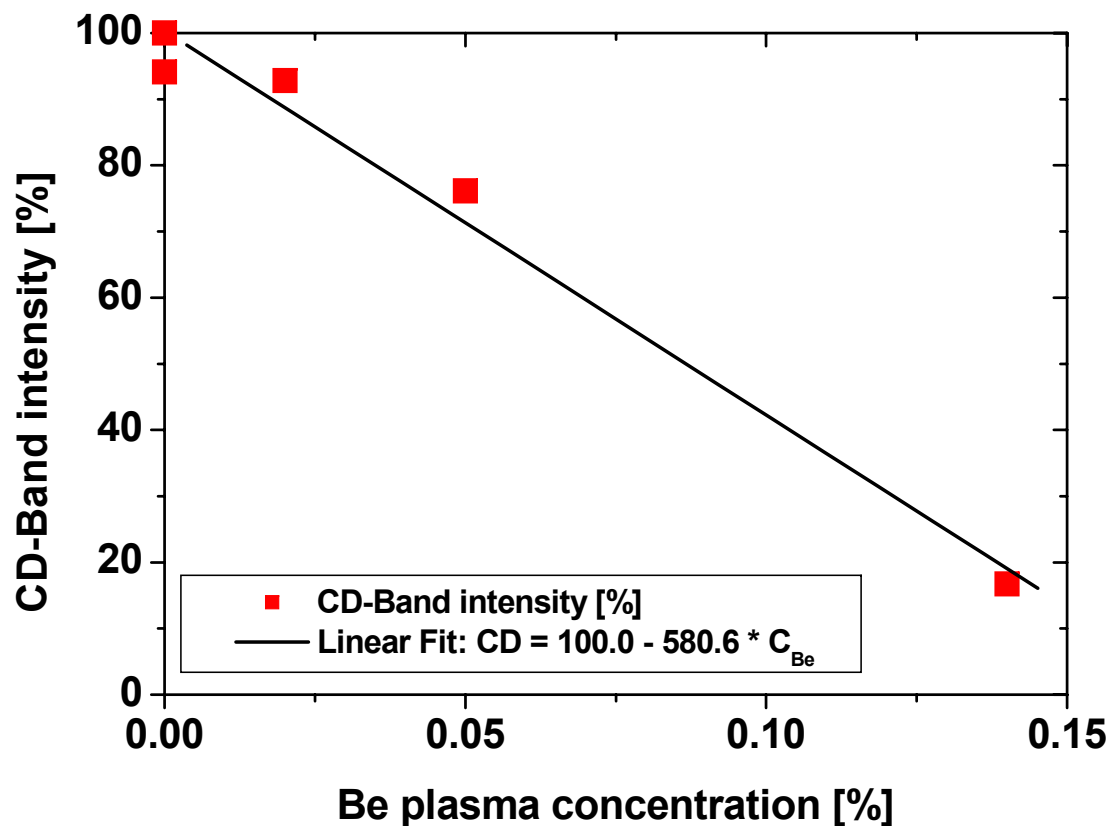
Carbon chemical erosion yields near the separatrix very low due to

- high temperature
- high fluxes
- low impact energy

Additional decrease of C-erosion by Be-deposition

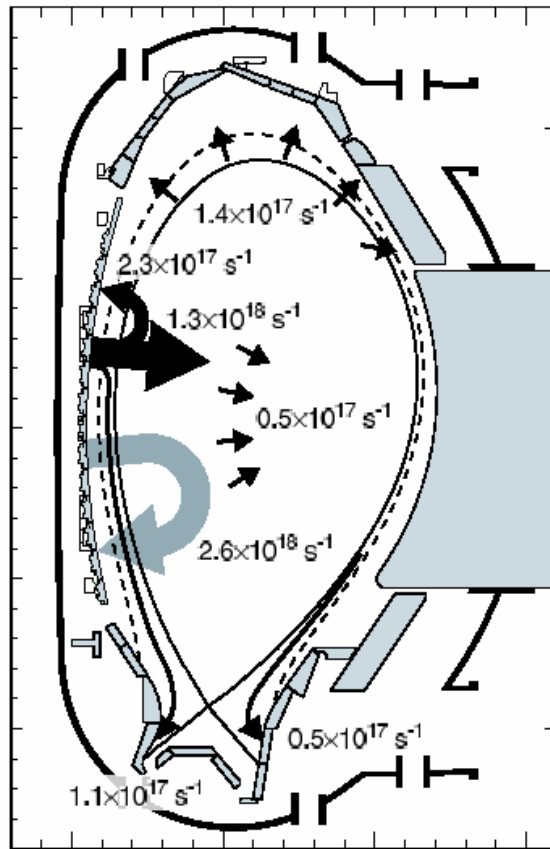


Results from Pisces linear plasma device



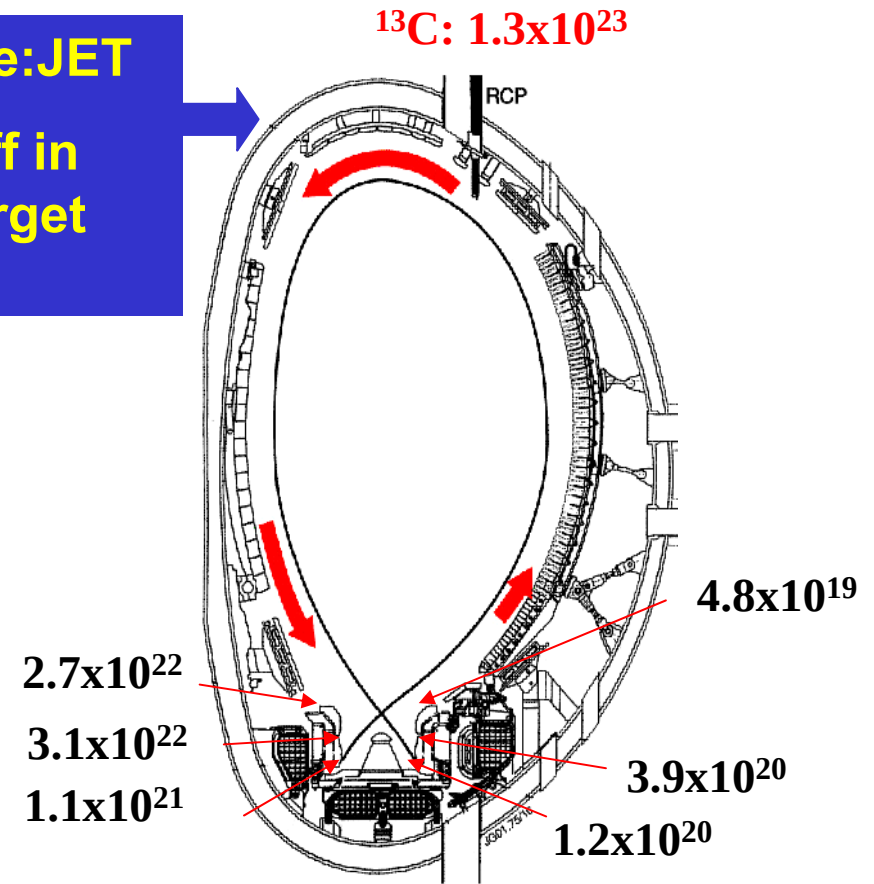
➤ CD-band decreases by ~80% for a Be plasma concentration of 0.15 %

➤ Be surface coverage after exposure 87% (AES).



AUG
W-migration

Example:JET
 $^{13}\text{CH}_4$ puff in
ohmic target
plasma

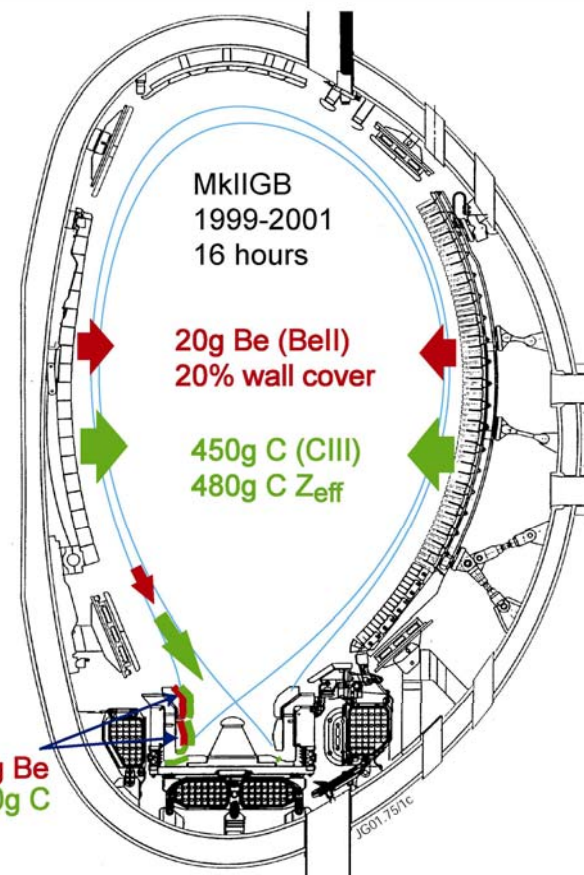
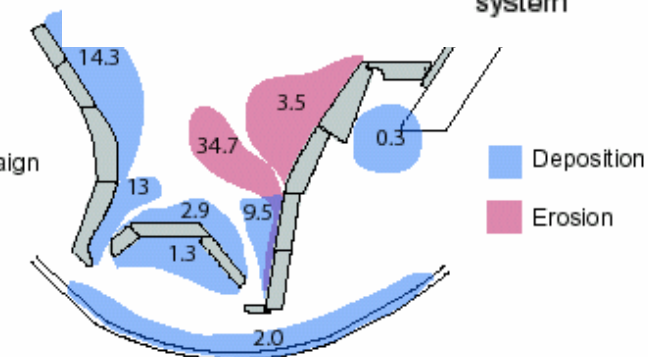
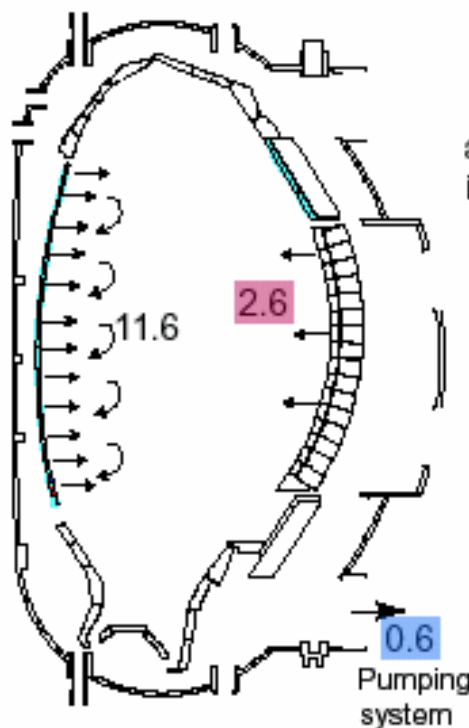


- preferential transport to the inner
- but significant deposition also in the outer divertor

- various other results (e.g. from Be deposition) indicate that transport is exclusively to the inner divertor

No uniform behaviour of erosion/deposition of outer divertor

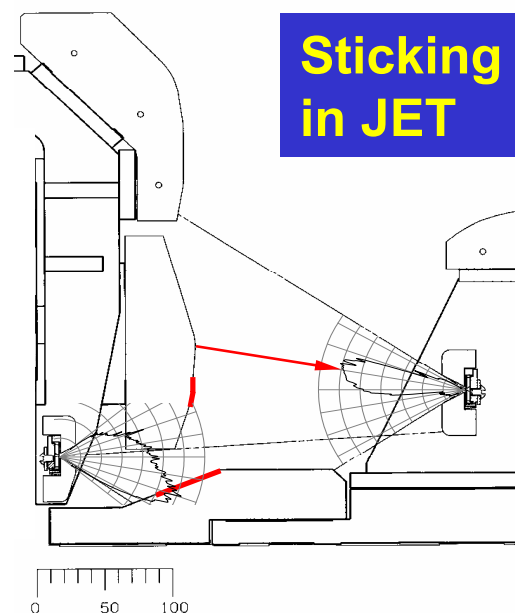
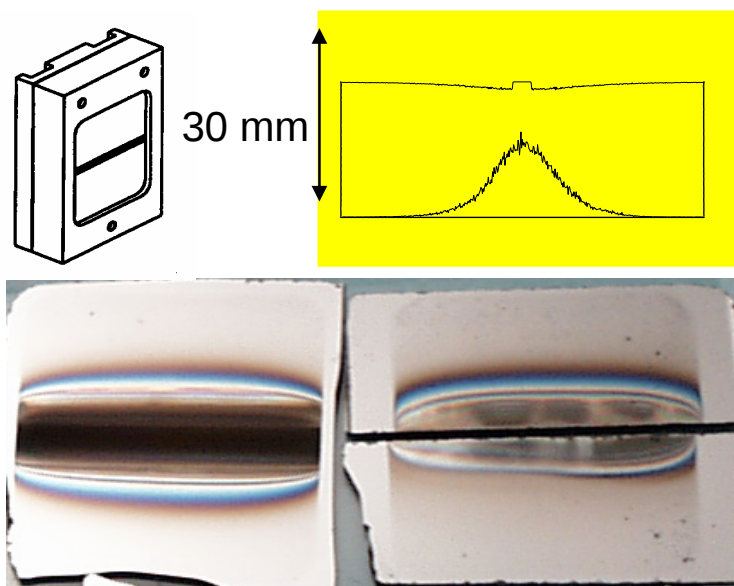
g/campaign



JET: balance between wall sources and inner divertor deposition, no contribution from outer divertor

AUG: balance needs large net-source in outer divertor or strong sources in main chamber not visible by spectroscopy

Major progress on characterisation of short range material migration



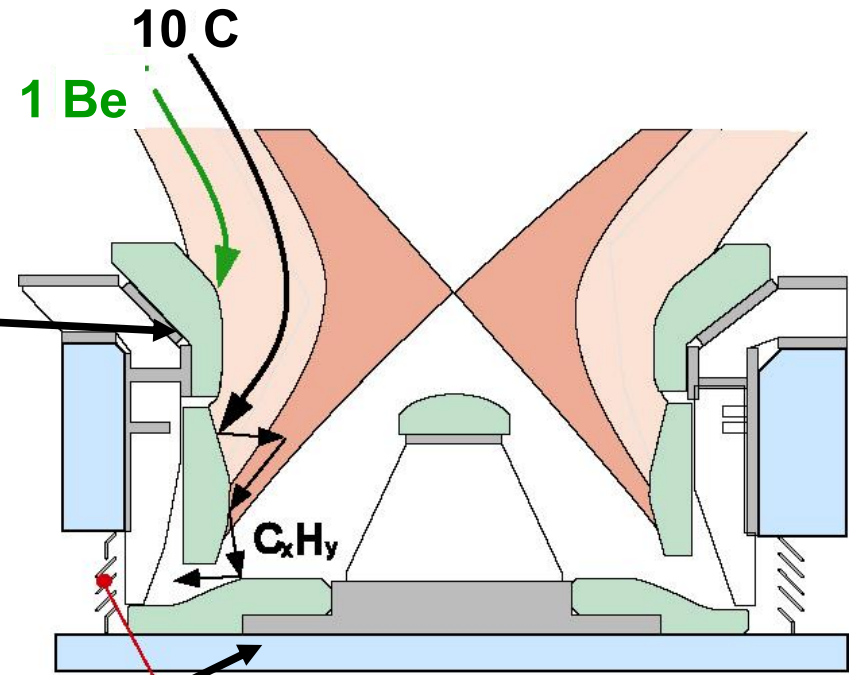
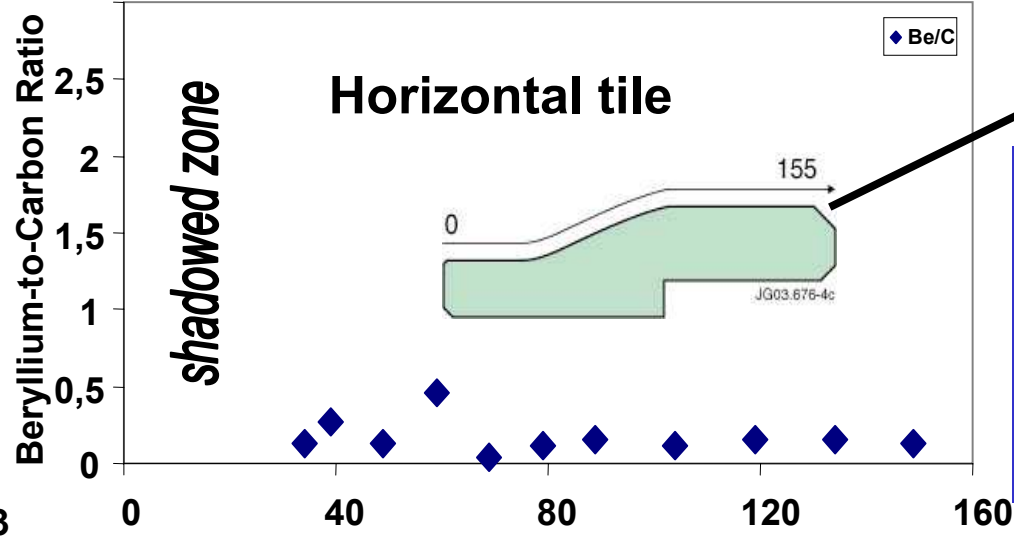
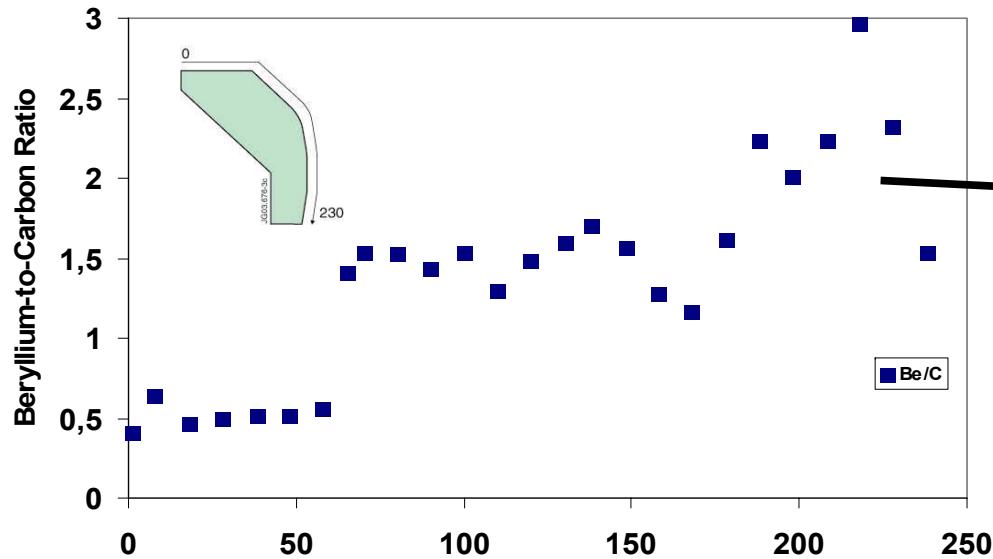
**Sticking Monitors
in JET**

- Majority of carbon (>99%) with high sticking (>0.9), small part (<1%) with low sticking probability (10^{-3}). The same conclusion from C deposition in TEXTOR and AUG pump ducts

Carbon deposition is largely line of sight from the place of origin

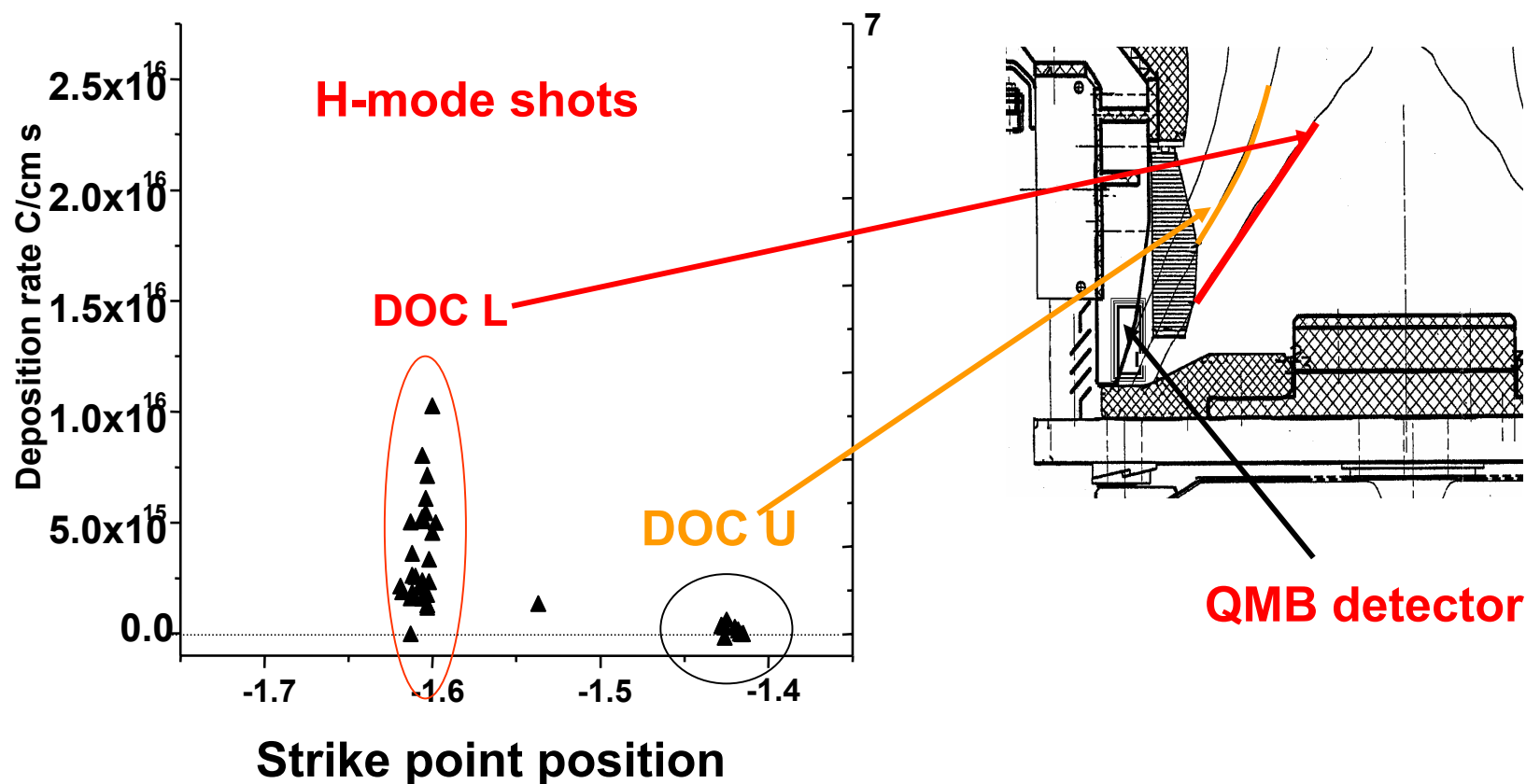
Post mortem analysis

Beryllium-to-Carbon Ratio on the Divertor Tile G1C



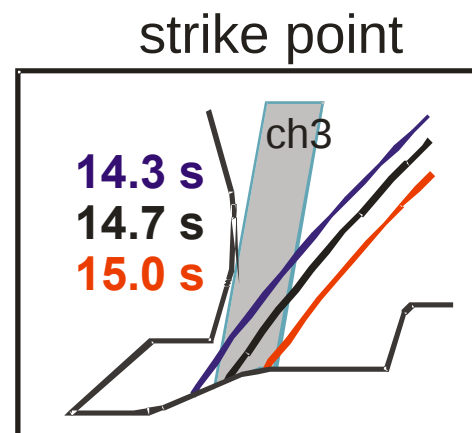
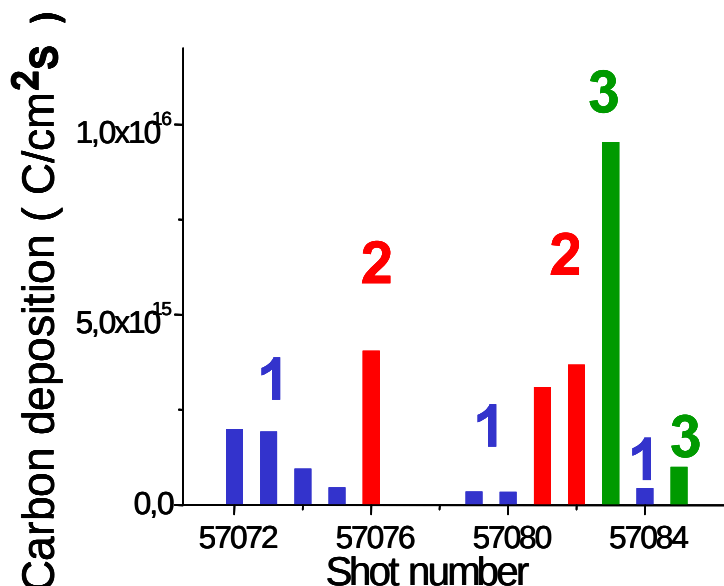
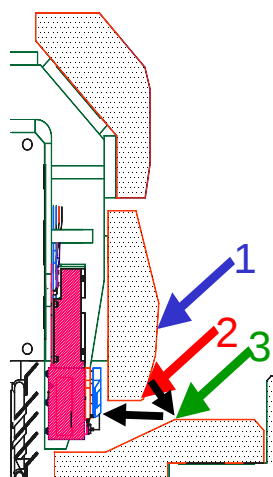
Louvers, QMB detector

- Carbon is transported further (chemical processes)
- Be stays at primary deposition

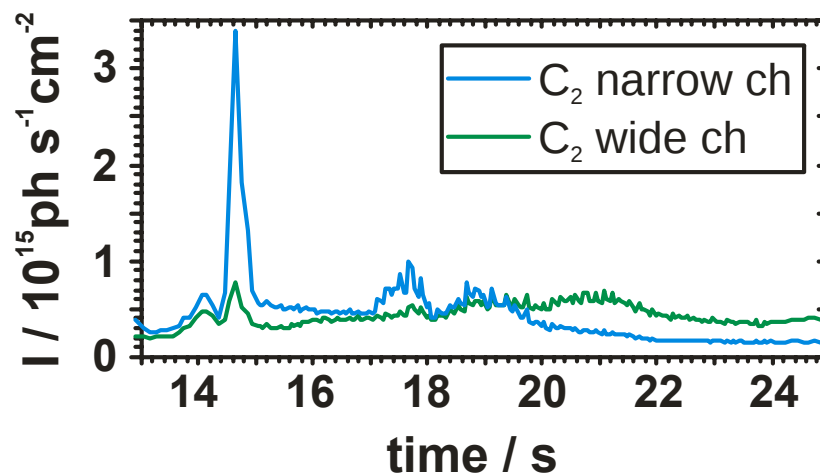


- Carbon deposition on QMB (louver) only for shots with strike point in the vicinity of louver entrance
- In agreement with ERO Code transport modelling
- Similar conclusions from AUG QMB data

12 Reproducible L mode shots

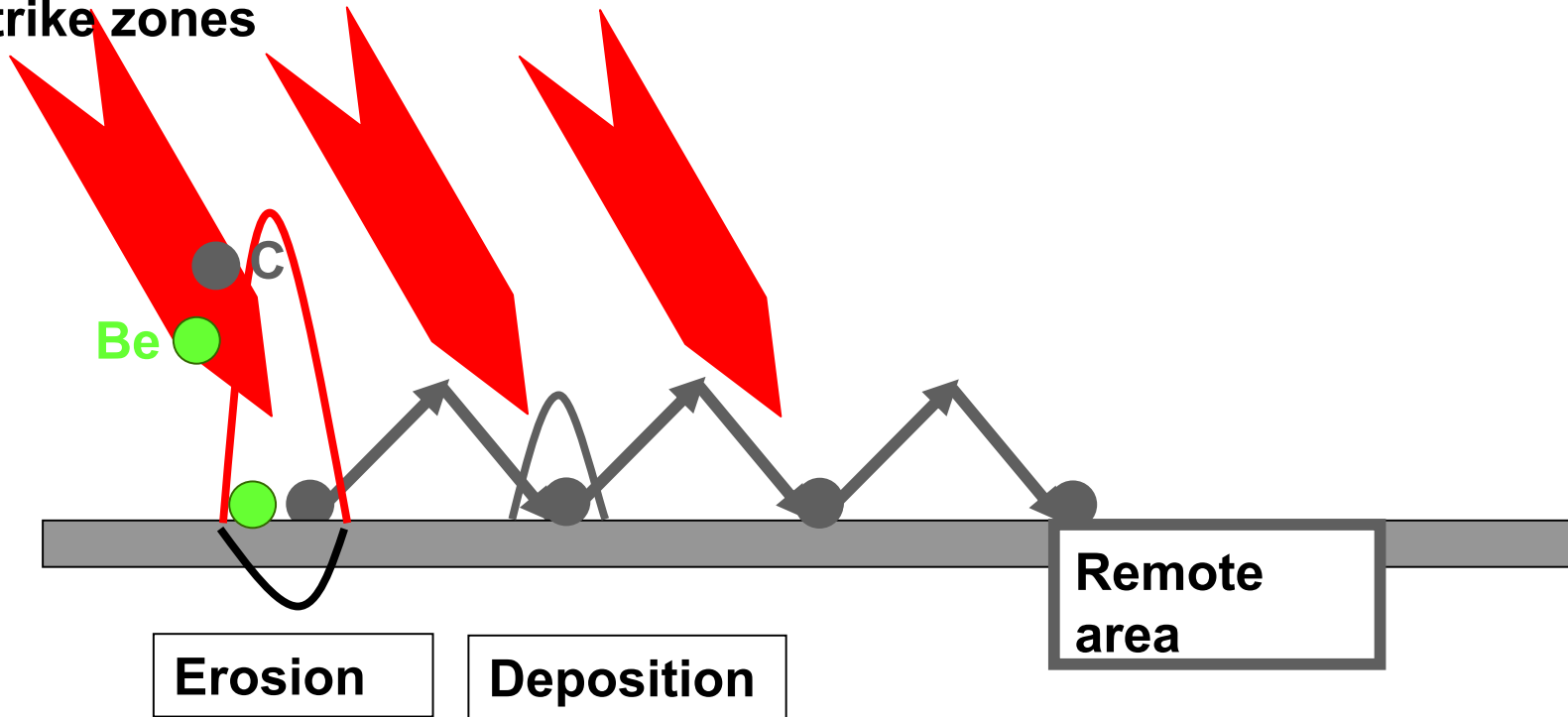


Spectroscopy



- A carbon layer is formed on the horizontal tile which is strongly eroded subsequent shots
- Carbon is transported stepwise to the louver region

Strike zones



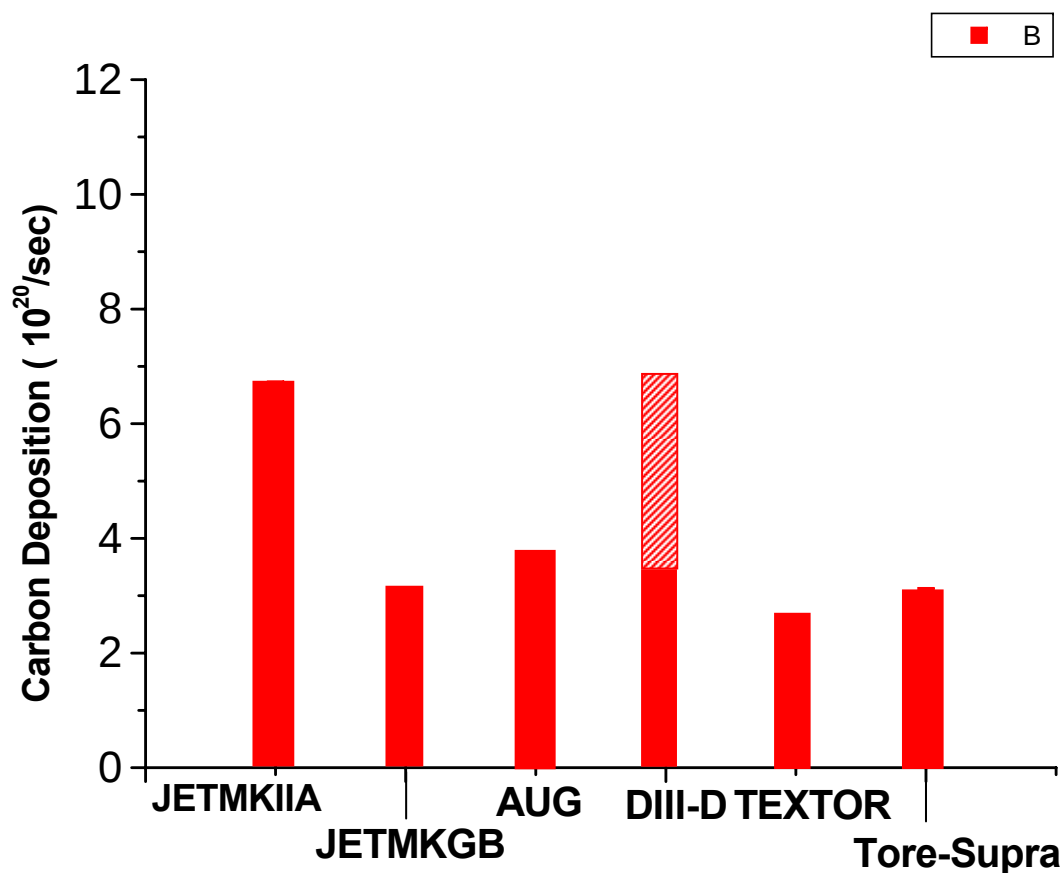
Short range C- transport near strike point by chemical erosion

No Be transport

Transport of C to remote areas by a combination of short range transport and varying plasma configurations

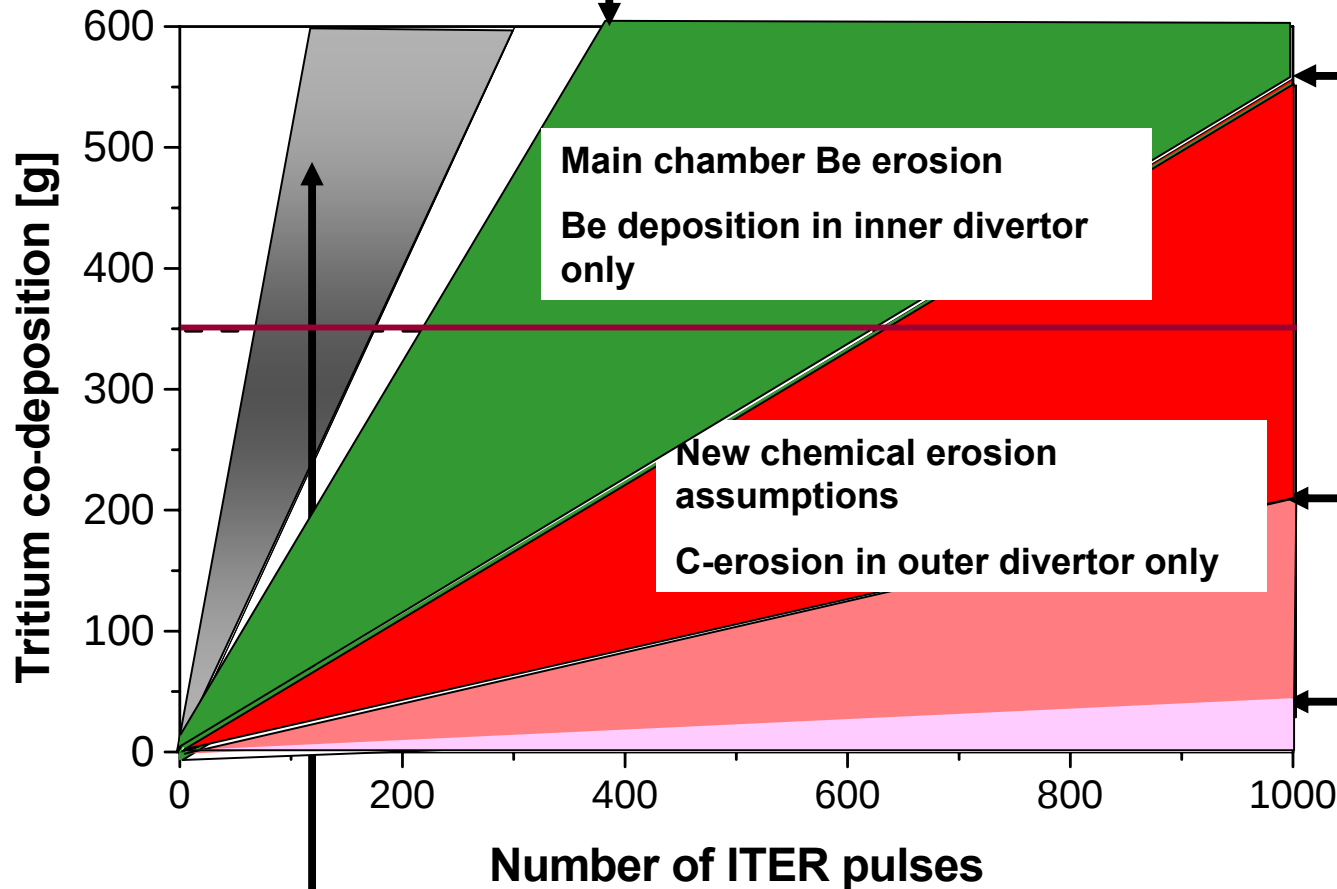
no Be transport

Pure empirical approach: similar campaign averaged carbon deposition rates in different machines, indicating some machine independence



S=0, moldyn, ERO
50gBe, D/Be= 0.1

Assumption: C-erosion in outer, Be deposition in inner leg



S=0, moldyn, ERO
6gBe, D/Be= 0.001

S=0.5, moldyn, ERO
6gBe, D/Be= 0.001

S=1, ERO
6gBe, D/Be= 0.001

Previous predictions 1% chemical erosion inner&outer, Brooks& Kirschner PSI 2002

A scatter of about 10^4 between worst and best assumptions !!

- Carbon chemical erosion sources in ITER are smaller than previously assumed, effect of Be deposition on erosion must be clarified further
- Main chamber is a net material source in general, but more analysis about mechanism and location in present machines is strongly needed
- Outer divertor behaviour is not uniform. Edge flows are not fully understood and predictions of erosion/deposition in the outer ITER divertor are difficult
- Due to chemical C-erosion, local C and Be migration in divertor differ largely, Be will be confined largely on plasma facing sides
- Tritium retention in ITER will be reduced (compared with extrapolation from carbon devices), depends largely on the T-inventory in Be-layers and is difficult to predict more precisely